Effect of diameter of nanoparticles and capture cross-section library on macroscopic dose enhancement in boron neutron capture therapy

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Abstract

Purpose: The aim of this study is evaluation of the effect of diameter of ¹⁰B nanoparticles and various neutron capture cross-section libraries on macroscopic dose enhancement in boron neutron capture therapy (BNCT).

Material and methods: MCNPX Monte Carlo code was used for simulation of a ²⁵²Cf source, a soft tissue phantom and a tumor containing ¹⁰B nanoparticles. Using ²⁵²Cf as a neutron source, macroscopic dose enhancement factor (MDEF) and total dose rate in tumor in the presence of 100, 200, and 500 ppm of ¹⁰B nanoparticles with 25 nm, 50 nm, and 100 nm diameters were calculated. Additionally, the effect of ENDF, JEFF, JENDL, and CENDL neutron capture cross-section libraries on MDEF was evaluated.

Results: There is not a linear relationship between the average MDEF value and nanoparticles' diameter but the average MDEF grows with increased concentration of ¹⁰B nanoparticles. There is an increasing trend for average MDEF with the tumor distance. The average MDEF values were obtained the same for various neutron capture cross-section libraries. The maximum and minimum doses that effect on the total dose in tumor were neutron and secondary photon doses, respectively. Furthermore, the boron capture related dose component reduced in some extent with increase of diameter of ¹⁰B nanoparticles.

Conclusions: Based on the results of this study, it can be concluded that from physical point of view, various nanoparticle diameters have no dominant effect on average MDEF value in tumor. Furthermore, it is concluded that various neutron capture cross-section libraries are resulted to the same macroscopic dose enhancements. However, it is predicted that taking into account the biological effects for various nanoparticle diameters will result in different dose enhancements.

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Key words: boron neutron capture therapy, cross-section library, dose enhancement nanoparticle.

Purpose

Boron neutron capture therapy (BNCT) was first described in 1935 [1]. Boron neutron capture therapy is a biochemically targeted radiation therapy based on the nuclear capture that happens when non-radioactive ¹⁰B is irradiated with thermal neutrons (0.025 eV). This interaction results in high linear energy transfer products (alpha particles and recoiling lithium-7 nuclei) [2]. These heavy particles deposit their energy (2.34 MeV) in range of 5-9 µm, which corresponds to cell diameter [3]. The destructive effects of these particles are restricted to boron containing cells. Since these heavy particles have very short path lengths in tissue, BNCT provides a way to selectively destroy cancer cells and spare adjacent normal cells [4]. Clinical applications of BNCT consist of treatment of gliomas [5], melanoma [6], liver cancer [7], colon cancer [8], sarcoma [9], lung cancer, mesothelioma, chest tumors [10], and head and neck tumors [11].

Despite recent improvements in conventional cancer treatment modalities, there is a significant need for new treatments that can destroy malignant cells while causing much less damage to the normal cells [12,13]. For this purpose, researchers are making effort to incorporate nanotechnology into present cancer therapeutics and imaging. The final goal of use of nanoparticles will be targeted delivery to the tumor, while causing to the least side effects for other tissues [14-16].

There are numerous studies, which have been carried out in the field of use of nanoparticles as dose enhancment media in radiotherapy. Some of these studies are reviewed herein: Ghorbani *et al.* [17] in a Monte Carlo study, have evaluated the effect of diameter of gold nanoparticles and method of modeling on macroscopic dose enhancement in brachytherapy. The results of their study indicated that with gold nanoparticles of larger diameter, the amount of macroscopic dose enhancement inside the tumor is higher.

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It has been also demonstrated that a simple model overestimates the dose macroscopic enhancement when compared with an exact model, in which nanoparticles are defined precisely in the Monte Carlo code. Pakravan *et al.* [18] in a Monte Carlo study have investigated the size effect of gold nanoparticles on macroscopic tumor dose enhancement in a 6 MeV photon beam. In the study, they found that there is a negligible relation between the dose enhancement and nanoparticles' diameter. It was also evident that for high energy photons, the macroscopic dose enhancement is less affected by the diameter of nanoparticles than their concentration.

Ghassoun et al. [19], in a Monte Carlo study, have evaluated dose distribution of a ²⁵²Cf brachytherapy source in BNCT. The results indicated that the magnitude of dose increase would depend on the distance from the ²⁵²Cf source as well as the concentration of ¹⁰B in the tumor cells. They found that a significant increase in dose is gained for tumor cells located at several centimeters away from the source with higher ¹⁰B loadings. Rivard and Zamenhof [20] carried out Monte Carlo calculations on a brain phantom to assess the boron-neutron capture dose and the ²⁵²Cf neutron energy spectra. ¹⁰B loadings up to 50 ppm did not noticeably modify the ²⁵²Cf fast neutron energy spectrum. For ¹⁰B concentrations lower than 50 ppm, the neutron dose emitted from ²⁵²Cf was small compared to the fast neutron sources. Finally, ²⁵²Cf could be clinically useful as a neutron source for boron neutron capture enhanced brachytherapy but would require the use of improved ¹⁰B agents with better loadings and excellent tumor selectivity.

Sumitani and Nagasaki [21] have studied BNCT assisted by boron-conjugated nanoparticles. In their study, they indicated that core-polymerized and boron-conjugated micelles administered via intravenous injection in tumor were averted almost entirely from the major organs, except for the tumor, after a week. Accordingly, boron-conjugated micelles indicated a promising approach for development of a new boron carrier for BNCT. Hwang et al. [22] have studied neutron capture nuclei containing carbon nanoparticles. In the study, they have demonstrated examples for neutron capture therapy (NCT) such as: BNCT, GdNCT, and CoNCT at the cellular level, in which ¹⁰B, ¹⁵⁷Gd, and ⁵⁹Co containing carbon nanoparticles could significantly induce acute cell deaths following receiving thermal neutron irradiation. The medical efficacies of the carbon nanoparticles can be better optimized by use of pure ¹⁵⁷Gd or ¹⁰B isotopes. The results indicated that these neutron capture elements containing carbon nanoparticles are very promising medical nanomaterials for destruction of tumors. In another NCT Monte Carlo study by Porras [23], the potential effect of enhancement of BNCT in presence of ³³S nanoparticles was investigated. The results showed that a noticeable local enhancement in dose could be obtained by means of sulfur nanoparticles. This effect may be synergic to the boron addition for NCT. This method can be considered as an alternative to the tailoring of neutron spectrum with the advantage of not increasing the gamma component of the background tissue dose.

Recently, in a report by International Atomic Energy Agency (IAEA) on various cross-section standards for neutron interactions, it was discussed that there are differences in neutron cross-section values at different neutron energies due to the related uncertainties in the measurements [24]. Since neutron capture cross-section libraries are used for calculation of neutron capture macroscopic dose enhancement, the use of different cross-section libraries should be investigated to answer this question that "has cross-section library an effect on macroscopic dose enhancement factor value?".

To the best of our knowledge, there is not any investigation on the effect of diameter of nanoparticles and neutron capture cross-section library on macroscopic dose enhancement in BNCT. In the present study, macroscopic dose enhancement for various diameters of ¹⁰B nanoparticles and various neutron capture cross-section libraries was evaluated and compared with each other.

Material and methods

Monte Carlo simulations were performed, while a ²⁵²Cf source [22,25] was defined as the source of neutrons in a soft tissue phantom. The source simulation was verified in a previous study on BNCT [26] and the same input files of the source simulations were used herein for calculation of macroscopic dose enhancement for various diameters of ¹⁰B nanoparticles (with 25, 50, and 100 nm diameters) with different concentrations (100, 200, and 500 ppm). Macroscopic dose enhancement for various neutron cross-section libraries (ENDF/B-VII.1, JEFF-3.2, JENDL-4.0u2 and CENDL-3.1) with 50 nm ¹⁰B nanoparticles was also calculated and compared via the simulations performed using MCNPX Monte Carlo code. Furthermore, macroscopic dose enhancement in presence of ¹⁰B nanoparticles for tumor distances of 1, 2, and 3 cm from the ²⁵²Cf source's center was investigated.

²⁵²Cf source geometry

A ²⁵²Cf (Applicator Tube [AT] model) source has been used as the source of neutrons [22,25]. AT ²⁵²Cf source model is composed of a primary capsule, a secondary capsule, and an active core. The cylindrical active element is made of Pd:Cf₂O₃ ceramic metal with 15.0 mm length and 1.23 mm diameter having mass density of 12.0 g/cm^3 . The primary capsule is composed of a Pt/Ir-10% alloy (with mass density of 21.51 g/cm³) with inner diameter of 1.35mm and outer diameter of 1.75 mm. The inner and outer lengths of the primary capsule are 15.50 and 17.78 mm, respectively. The secondary capsule has the same composition but its inner diameter is 1.80 mm and its outer diameter is 2.80 mm, while its inner and outer lengths are 17.82 mm and 23.14 mm, respectively. The inner and outer capsules have rounded with weld ends. There is a Bodkin eyelet with 0.635 mm diameter in the outer capsule [22]. Figure 1 illustrates the geometry of this source model.

²⁵²Cf source simulation verification

In this study, the validated simulations of AT ²⁵²Cf source presented in reference [26] were used. While



Fig. 1. Geometry of applicator tube ²⁵²Cf source used in this study

the validations were not repeated in the present study, a summary of the source simulation in reference 26 is presented. MCNPX (version 2.6.0) [27] Monte Carlo code has been used for simulation of the ²⁵²Cf source. Dosimetric parameters of the source, including air kerma strength, dose rate constant, and radial dose function have been calculated based on the formalism presented in the updated report by American Association of Physicists in Medicine (AAPM) Task Group No. 43 (TG-43U1) [28].

In addition to TG-43U1 parameters, total dose rate per microgram of the 252 Cf source (cGy/(h-µg)) has been also calculated. Total dose equals to the sum of dose contributions from primary photons from ²⁵²Cf, neutrons and secondary photons induced from neutron interactions. The photon spectrum of the ²⁵²Cf source has photon energies in the range of 10 keV-10 MeV [29]. One microgram of 252 Cf source emits 2.314 × 10⁶ fast neutrons/s [30] and also has a photon yield of about 1.3×10^7 photons/s [31]. Maxwellian fission distribution has been utilized as the distribution for the neutrons emitted by the ²⁵²Cf source. The neutron spectrum emitted by ²⁵²Cf source has an average energy of 2.1 MeV and also the most probable energy of ~0.7 MeV [32]. In calculation of dose rate constant, radial dose function, and total dose rate, a water phantom with 15 cm radius was defined. F6 tally was applied to score the photon and neutron doses in the verification step. For calculation of the total dose, two input files have been run: the first input file included a photon source, defined with the photon spectrum emitted by ²⁵²Cf source, and the photon tally was computed for the purpose of calculation of the primary photon dose component; the second input file included the neutron spectrum of the ²⁵²Cf source and the dose components from the neutrons and secondary photons were calculated. In calculation of dose rate, constant and radial dose function, a number of 10⁷ neutrons were scored and the maximum type A uncertainty in Monte Carlo calculations was 0.78%. In calculation of total dose rate, a number of 2.0×10^8 particles were run in each input file. The type A uncertainty in the tally cells in these Monte Carlo calculations were less than or equal to 6.60% in these two types of files. Finally, in the study by Khosroabadi et al. [26], the comparison of air kerma strength, dose rate constant, radial dose

function, and total dose rate has shown good agreement between the obtained data and those reported in other studies [32-34] for the AT ²⁵²Cf source, which validated the simulations for this source model.

Effect of diameter of nanoparticles and capture cross-section library

In this step, the verified AT ²⁵²Cf source simulations from reference 26 were used for calculation of macroscopic dose enhancement for various diameters of ¹⁰B nanoparticles and various neutron cross-section libraries. MCNPX (version 2.6.0) Monte Carlo code has been utilized for this purpose. Soft tissue composition announced by the report No. 44 of International Commission on Radiation Units and Measurements (ICRU), with weight fractions of H: 0.102; C: 0.143; O: 0.708; N: 0.034; Na: 0.002; P: 0.003; Cl: 0.002; S: 0.003; K: 0.003, and mass density of 1.06 g/cm³ has been used as the phantom media [35]. Furthermore, abundances of natural isotopes of each element have been utilized in these dosimetric calculations. The soft tissue phantom had a spherical form with 15 cm radius, while the ²⁵²Cf source has been defined, including its capsules and active core, at the center of the phantom. The phantom contained a 1 cm × 1 cm × 1 cm tumor placed at 1 cm from the ²⁵²Cf source's center in the transverse direction relative to the source's longitudinal axis. 100, 200, and 500 ppm concentrations of ¹⁰B has been considered as the capture agent inside the tumor in the form of nanoparticles. The effect of diameter of ¹⁰B nanoparticles on macroscopic dose enhancement was evaluated. For this purpose, 25, 50, and 100 nm diameters of ¹⁰B nanoparticles were studied. Each nanoparticle diameter and concentration has been examined in a separate simulation. The composition of the tumor included the capture medium (¹⁰B nanoparticles) and soft tissue, and was defined as distribution of the nanoparticles in the Monte Carlo programs. ¹⁰B nanoparticles were defined as nanospheres with 25, 50, and 100 nm diameters using lattice cards in the MCNPX code. In this code, a lattice can be utilized as rectangular parallelepiped (RPP) surfaces for definiton of repeated structures inside a volume. ¹⁰B nanoparticles were distributed uniformly inside

the tumor volume. Our rational for selection of 25, 50, and 100 nm diameters of nanoparticles was that nanoparticles ranging in diameter from 1 nm to 100 nm are usually applied in the field of nanomedicine [36]. By considering the diameter and concentration of ¹⁰B nanoparticles, the tumor was divided into cubic cells with dimensions of order of nanometer and in each cell a ¹⁰B nanoparticle was defined. The dimensions of the tally voxels were 2 mm × 2 mm × 2 mm. Since the tumor was considered as a 1 cm × 1 cm × 1 cm cube, there were 5 voxels on the transverse line in the tumor. In this study, the total dose was calculated as the sum of the dose components from primary and secondary photons, neutrons, and the capture products. For calculation of dose component from neutron capture products in each voxel, the neutron flux in each voxel inside the tumor was calculated using F4 tally and then was converted to dose by multiplication of the neutron flux and kerma factor. The kerma factors of 10 B(n, a) interactions have been obtained from the following formula using the neutron capture cross-sections (σ):

$$F_{n(E_{i)}} = \sum_{j,k} \sigma_{k(j)} \frac{xi}{M_{j}} N_A \varepsilon_{k,j}$$
⁽¹⁾

where F_n is kerma factor; x_i is the mass fraction of the capture agent; M is atomic mass; N_A is Avogadro's number; and ε is the delivered energy in the capture process.

For each nanoparticle diameter, two input files have been run: the first input file with a photon source (emitted by ²⁵²Cf) and photon tally scoring for the purpose of calculation of the primary photon dose component; the second input file with a neutron source (emitted by ²⁵²Cf) and the dose components from secondary photons, neutrons, and capture products were scored. The first and second input files were run for a number of 10⁸ particles in calculation of macroscopic dose enhancement and the maximum Monte Carlo type A uncertainty over all the situations and tally cells was 4.67%. Macroscopic dose enhancement in the center of each voxel in tumor on transverse line was calculated and then macroscopic dose enhancement values were averaged. Macroscopic dose enhancement factor (MDEF) in each voxel has been determined as the total dose in the voxel with presence of ¹⁰B nanoparticles in tumor to the total dose in the same voxel without presence of ¹⁰B nanoparticles. Since the absorbed dose in radiotherapy is a macroscopic variable, it was preferred in the present study to use a macroscopic dose enhancement defintion, as it is in a number of other studies in this field. However, the nanoparticles were defined in a nanometer scale in the form of nanosphers. ENDF/B-VII.1 cross-section library

Table 1. The average macroscopic dose enhancement factor values for 25, 50 and 100 nm ¹⁰B nanoparticles with concentrations of 100, 200, and 500 ppm

	25 nm	50 nm	100 nm
100 ppm	1.02	1.03	1.04
200 ppm	1.04	1.05	1.03
500 ppm	1.10	1.08	1.10

was used in the evaluation of the effect of nanoparticle's diameter with different concentrations.

The effect of neutron capture cross-section library for neutron-boron capture interaction on MDEF for various library sources including: ENDF/B-VII.1, JEFF-3.2, JENDL-4.0u2, and CENDL-3.1 was evaluated. The capture cross-sections have been elicited from the cross-section data reported by the International Atomic Energy Agency (IAEA) webpage for (n, α) interaction with ¹⁰B isotope [37]. Macroscopic dose enhancement factor value and total dose rate $(cGy/(h \cdot \mu g))$ were calculated for the cross-section libraries for 500 ppm concentration of 50 nm diameter ¹⁰B nanoparticles. The details of these simulations (eg. the number of input files, the number of particle histories, etc.) were similar to those mentioned above in the evaluation of the effect of diameter of nanoparticles. The maximum Monte Carlo type A uncertainty over all the situations and tally cells was 4.56%.

Beside to the distance of 1 cm for tumor from the source center, other distances (2 cm and 3 cm) were examined as well. For this purpose, the tumour was defined at distances of 2 cm and 3 cm from the center of the 252 Cf source at the transverse direction, while each distance was defined in a separate simulation. In these two cases, the diameter of the nanoparticles was 50 nm and the concentration was 500 ppm. Other simulation details were the same as that described above for 1 cm tumor distance.

Macroscopic dose enhancement in presence of 500 ppm 10 B nanoparticles with 50 nm diameters for tumor distances of 1, 2, and 3 cm from the 252 Cf source's center was investigated as well. For distances of 2 cm and 3 cm, 4×10^8 particles were scored in each input program and the maximum type A uncertainty in the calculations for these programs was 4.96%.

Results

The average MDEF values for 25, 50 and 100 nm ¹⁰B nanoparticles with concentrations of 100, 200 and 500 ppm are listed in Table 1. The average values have been obtained as the averages of MDEFs over the five voxels on the transverse line inside the tumor.

The plots of MDEF and the total dose rate values (cGy/ $(h \cdot \mu g)$) inside the tumor versus radial distance from the ²⁵²Cf source are shown in Figure 2 for different diameters of ¹⁰B nanoparticles. In this figure, parts (A) and (B) are related to MDEF and total dose rate, respectively.

The average MDEF values for ENDF/B-VII.1, JEFF-3.2, JENDL-4.0u2, CENDL-3.1 capture cross-section libraries for 500 ppm concentrations of ¹⁰B with 50 nm diameter was obtained equal to 1.08. This value was obtained the same for the four capture cross-section libraries evaluated. The plots of the MDEF and the total dose rate values (cGy/($h \cdot \mu g$)) inside the tumor were illustrated in Figure 3 for the four capture cross-section libraries. In these plots, parts (A) and (B) correspond to MDEF and total dose rate, respectively.

For tumor distances of 1, 2, and 3 cm from the ²⁵²Cf source's center, the average MDEF values for 50 nm ¹⁰B nanoparticles with 500 ppm concentration were obtained equal to 1.08, 1.18, and 1.27, respectively.



Fig. 2. Macroscopic dose enhancement factor (part A) and total dose rate (part B) in tumor for 25, 50, and 100 nm ¹⁰B nanoparticles with 500 ppm concentration of ¹⁰B nanoparticles (with ENDF capture cross-section library)



Fig. 3. Macroscopic dose enhancement factor (part A) and total dose rate (part B) in tumor for ENDF, JEFF, JENDL, and CENDL capture cross-section libraries with 500 ppm concentration of ¹⁰B nanoparticles (with 50 nm diameter)

The percentage contributions of each dose component (primary photons, secondary photons, neutrons, and boron capture products) from the total dose in the central voxel inside the tumor without and with 500 ppm concentration of ¹⁰B nanoparticles are listed in Table 2.

Discussion and conclusions

In this study, the effect of diameter of ¹⁰B nanoparticles with different concentrations and the effect of various neutron capture cross-section libraries on macro-

Table 2. The percentage contributions (%) of dose components of the total dose without and with 500 ppm boron nanoparticles in tumor. The values are related to the central voxel in the tumor

	Primary photon	Secondary photon	Neutron	Boron capture products
Without nanoparticles	29.45	0.80	69.75	-
25 nm nanoparticles	27.75	0.75	64.05	7.44
50 nm nanoparticles	27.59	0.75	64.56	7.10
100 nm nanoparticles	27.21	0.76	65.24	6.79

scopic dose enhancement in BNCT were investigated. The obtained results (Table 1) indicates that various ¹⁰B nanoparticle diameters have no dominant effect on average MDEF value in tumor in various concentrations. On the other hand, the data in this table imply that the average MDEF values grows with increase of concentration for ¹⁰B nanoparticles with same diameter. This effect confirms the results of a previous study in this field on BNCT with various ¹⁰B nanoparticles [26]. It should be noticed that from only physical point of view, various nanoparticle diameters have no dominant effect on average MDEF, however, for detailed evaluation of the effects of nanoparticle diameters, biological effects should also be considered. For the purpose of incorporation of biological effect, it is normal that each dose component (from photons, neutrons, and capture products) is multiplied with the corresponding relative biological effectiveness (RBE) of that particle. Various RBE values have been reported for photons and neutrons in literatures. As an example, values of 1.0 and 3.0 were reported by Kawabata et al. for photons and neutrons, respectively [38]. Various compounds were proposed for delivery of boron to tumor: borono-phenylalanine (10BPA) and sodium mercapto undecahydro-dodecaborate (Na $_2^{10}B_{12}H_{11}SH:BSH$). The distribution of these compounds is not homogeneous in tissue. Therefore, another factor is applied in dose calculations in the field of BNCT, which is compound biological effectiveness (CBE). This factor incorporates the effect of both RBE and boron distribution in the dose calculations. Kawabata *et al.* [38] reported a CBE of 3.8 for the ${}^{10}B(n, \alpha)$ -⁷Li reaction in BPA. This value is specific to BPA and cannot be used for other compounds. Since the RBE and CBE for the neutron-alpha interactions in the ¹⁰B nanoparticles used in this study are not available, the MDEFs have been only calculated for the physical dose values. Macroscopic dose enhancement factor obtained from the product of the CBE and the physical dose will differ with the MDEF calculated from the physical dose alone. It is obvious that the RBE weighted MDEF values will depend on the RBE values, which are used for various nanoparticle diameters in the calculations. Since various nanoparticle diameters have different cellular penetration, therefore they have different distributions and RBEs. In our study, one notes that the RBE values of ¹⁰B nanoparticles with various diameters are not available, therefore only physical dose enhancements were calculated for different diameters of nanoparticles. It is predicted that taking into account the biological effects for various nanoparticle diameters will result in different macroscopic dose enhancements when compared to those obtained herein.

Considering the effect of diameter of nanoparticles and biological aspects on macroscopic dose enhancement in other radiotherapy modalities may be useful. According to the definition by American Society for Testing and Materials standard, nanoparticles are particles ranging in diameter from 1 to 100 nm in two or three dimensions [36]. It has been shown that macroscopic dose enhancement inside a tumor is influenced by the beam's quality, concentration, diameter of nanoparticles, etc. [18].

The overall particle's size must be large enough to avoid renal clearance but small enough to evade uptake by reticuloendothelial system [39]. It has also been shown that for nanoparticles smaller than 40 nm in diameter, both the half-life and biodistribution of nanoparticles are influenced by the coating material rather than the mean size [40].

In experimental studies on GNPs, the size of GNPs has been shown to be as a factor effecting on radiosensitization [41]. Literatures have reported that smaller GNPs are more effective [42], while the others suggest an opposite effect [43]. In an in vitro study, Chithrani et al. investigated the impact of GNP diameter, radiation energy, and concentration on radiosensitisation in Hela cells. It has been evident that GNPs with 50 nm diameter have higher radiosensitization enhancement factor (REF) than 14 nm or 74 nm GNPs, although the highest amount of gold was delivered with 74 nm nanoparticles [44]. The reason for this effect was due to higher gold loading in nanoparticles with larger particle sizes [45]. A smaller diameter GNPs (15 nm) showed higher distribution in tissues compared to larger particle diameters. Hillyer and Albrecht [46] evaluted the gastrointestinal uptake and tissue/organ distribution of 4, 10, 28, and 58 nm diameter GNPs. The smallest size GNPs (with 4 nm diameter) showed greater uptake in various tissues. Gold was not detected in most of the tissues evaluated following oral administration of 58 nm diameter colloidal gold. In a study by Jiang *et al.* [47] it was confirmed that the optimal diameter for nanoparticle cellular entry was 40-50 nm.

Additionally to the above studies, there are also Monte Carlo studies carried out in the field of dose enhancement by GNPs. Ghorbani et al. [17] evaluated the effect of diameter of gold nanoparticles with 50, 100, and 200 nm diameters on macroscopic dose enhancement in brachytherapy with photon sources. Their results indicated that with gold nanoparticles of larger diameter, the amount of macroscopic dose enhancement is higher inside the tumor. In another study, Pakravan et al. [18] investigated the effect of size of nanoparticles with 25-200 nm diameters on macroscopic dose enhancement in tumor with a 6 MV photon beam. It has been found that there is a negligible relation between the macroscopic dose enhancement and nanoparticle's diameter in this photon energy. It was evident that for high energy photons, the macroscopic dose enhancement is lower affected by the diameter of nanoparticles than their concentration. In a study by Leung et al. [48], the interaction of X-rays with GNPs with 2, 50, and 100 nm diameters has been simulated, and the results showed that nanoparticles with larger diameters provide greater dose values. Lechtman [49] carried out a Monte Carlo study on the role of nanoparticle's diameter, photon energy, intracellular location, and concentration of nanoparticles on the radiosensitization to determine the optimal clinical use of GNPs. He found that GNPs' size can strongly impress radiosensitization due to the size dependent cellular uptake. Similarly Chithrani et al. [44] found that GNPs of 50 nm were a desired size for accumulation and retention within Hela cells.

In a clinical study, Mortensen *et al.* [50] used boron carbide nanoparticles as a novel agent in T cell-guided BNCT. They found that desired size distribution for boron carbide nanoparticles is 73 nm. The above mentioned

effects are related a combination of photon sources and gold nanoparticels. It is obvious that they may not be extended simply for a neutron source and ¹⁰B nanoparticles in BNCT applications. However, similar phenomena may be expected in some cases in application of ¹⁰B nanoparticles in BNCT. In other words, additionally to the physical macroscopic dose enhancement seen for ¹⁰B nanoparticles, other biological aspects should be sought and evalualted as well for ¹⁰B nanoparticles with various diameters.

While the MDEF values have different decreasing or increasing trends in the tumor for various diameters of nanoparticles and capture cross-section libraries (Fig. 2 and 3), the dose rate values (cGy/($h \cdot \mu g$)) are decreasing in tumor with the radial distance from the ²⁵²Cf source. This decreasing trend in dose rate is due to inverse square law with distance and the attenuation in phantom material. It can be concluded from this effect that in treatment planning in neutron capture therapy, various points inside a tumor will experience different macroscopic dose enhancements and this effect should be considered. However, it is clear that MDEF differs from dose rate value and the MDEF only cannot be the sole criterion in clinical considerations.

The average MDEF values were the same for various capture cross-section libraries (for 500 ppm concentration of ¹⁰B with 50 nm diameter the MDEF was obtained equal to 1.08). Furthermore, the trends of the MDEF values inside tumor are the same for different capture cross-section libraries (Fig. 3). In summary, while there may be local differences in various neutron capture cross-section libraries, it has been shown that using different cross-section libraries in BNCT calculations do not show a difference in MDEF and total dose rate values.

The average MDEF values for tumor distances of 1, 2, and 3 cm from the ²⁵²Cf source's center are 1.08, 1.18, and 1.27, respectively. As it can be evident from these data, there is an increasing trend for average MDEF with the tumor distance. This effect may be due to the change in the energy spectrum of the neutrons (or photons) with the distance in the phantom. It is obvious that when the neutrons (and photons) emitted by the $^{252}\mbox{Cf}$ source traverse in the phantom, the interactions between the particles and the phantom material will change the energy spectrum. Therefore, the number of thermal neutrons will be different at various distances from the source. This will effect on the number of capture interactions with ¹⁰B nanoparticles at various distances. A tumor at furthere distance from the source experience a higher average MDEF and this will be an advantage for this type of tumor in BNCT. On the other hand, the dose is different from MDEF and in this case the normal tissue between the source and tumor receives a larger dose due to the steep dose gradient in the vicinity of the source.

Based on the data in Table 2 (for concentration of 500 ppm), the maximum and minimum doses that will effect the total dose rate in tumor are neutron and secondary photon doses, respectively. Furthermore, according to these data it can be concluded, while there was not a linear relationship between the nanoparticles' diameter and the MDEF value, for a larger diameter of nanoparticles the boron-related dose component reduces in some extent. It

can be also mentioned that in BNCT calculations, the total dose should be involved and considering only the neutron dose will introduce underestimations in dose calculations.

According to the results of this study, it can be concluded that the selection of optimum diameter of ¹⁰B nanoparticles needs to additional studies on biological aspects. In other words, it is necessary to take into account the biological effects differences in dose calculations. For this purpose, each dose component should be multiplied with the corresponding RBE of each particle. It can be also concluded that using various capture cross-section libraries result to the same average MDEF values.

There are other limitations in the present study. Lack of RBE for various diameters of ¹⁰B nanoparticles is one of the limitations. A radiobiologic study on this subject can be a basic support for dose enhancement studies on various ¹⁰B nanoparticle diameters. Furthermore, a uniform distribution of ¹⁰B was defined in the tumor, while in a real situation the distribution will be nonuniform. The obtained macroscopic dose enhancement results are geometry-specific and it is obvious that by having a different source and tumor geometry, the results will be different in some extent. Since absorbed dose is a macroscopic variable, it was preferred to use 2 mm × 2 mm × 2 mm voxel sizes in calculations of dose enhancements. This method was used in a number of previous studies in the field of dose enhancment in radiotherapy [17,18,26]. On the other hand, Tsiamas et al. [51] and Zygmanski et al. [52] showed that the nanoscopic dose enhancement factor is practically unity after 1-2 microns. Using a voxel of 2 mm × 2 mm × 2 mm means that the calculated dose enhancement factor is lower than that it is in the reality as the total dose to soft tissue would not be smaller than the dose when nanoparticles are present. Therefore, performing a similar study, as it is in the present one, on dose enhancment by various diameters of ¹⁰B nanoparticles at a micro scale will be as a complementary to the present results. This is especially true for a BNCT study as the boron capture products have short ranges in soft tissue. Another aspect of the present study, which should be considered in more details, is the definition of concentration in evaluation of the effect of nanoparticle's diameter. Many studies have used concentration but the calculated dose enhancement factor for these studies is more a macroscopic dose rather than a nanoscopic one. In the nanoscopic scale, quantities such as concentration, dose etc. have different meanings. Furtheremore, the AT ²⁵²Cf source is a low dose rate neutron source and primary sources with higher neutron fluxes can provide higher number of neutrons and therefore higher dose enhancment values. These items can be as subjects of further research in the field of BNCT.

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Disclosure

Authors report no conflict of interest.

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384

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